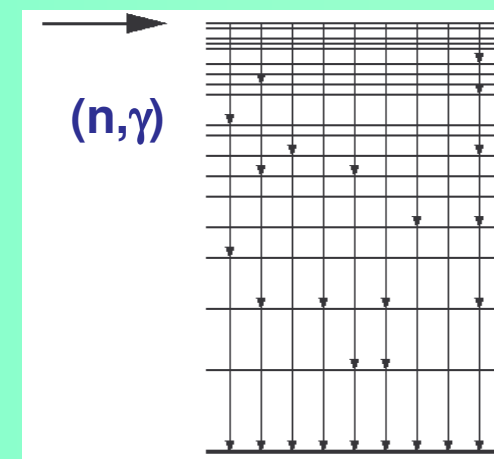
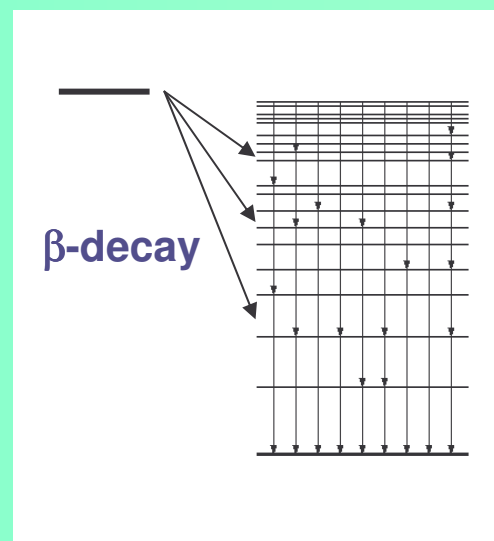
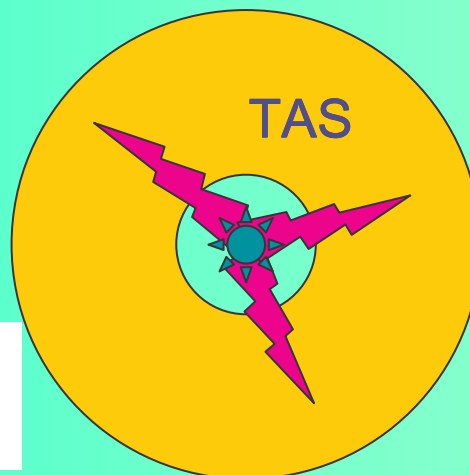


# Neutron MC simulations for Total Absorption Gamma-ray Spectroscopy

- Total Absorption Spectroscopy is the best method to measure beta strengths in  $\beta$ -decay (the only valid one far from stability)
- It is also a powerful method to measure neutron capture cross-sections (the only useful for rare or radioactive samples)
- A major source of systematic error is contamination/background signals

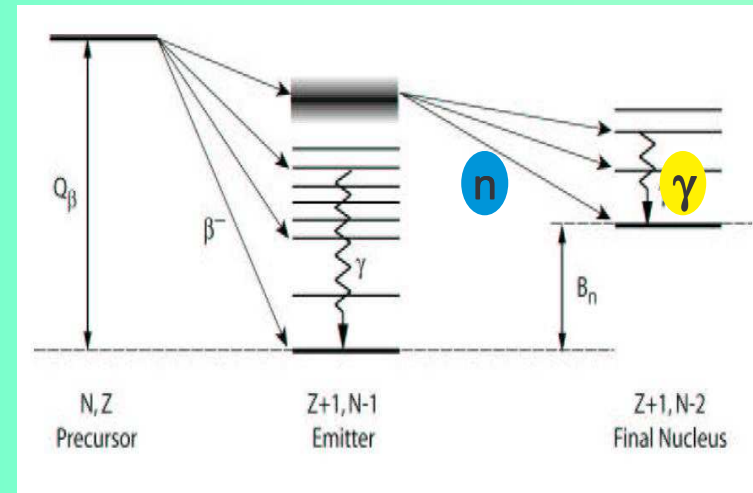
TAS: large  $4\pi$  scintillation detector



- A particular **challenge** is the application of the TAS technique in  $\beta$ -decay at the neutron rich side, due to the **beta delayed neutrons**

The beta-delayed **neutrons** and the subsequently emitted **gamma-rays** (may) become a **contamination source**

**Grand-daughter  $\gamma$ -rays** are **prompt** with daughter  $\gamma$ -rays  
 → solution: “subtract” from data



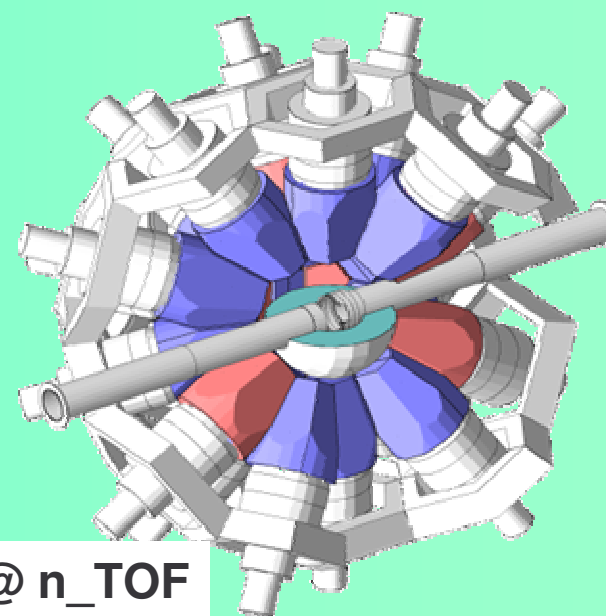
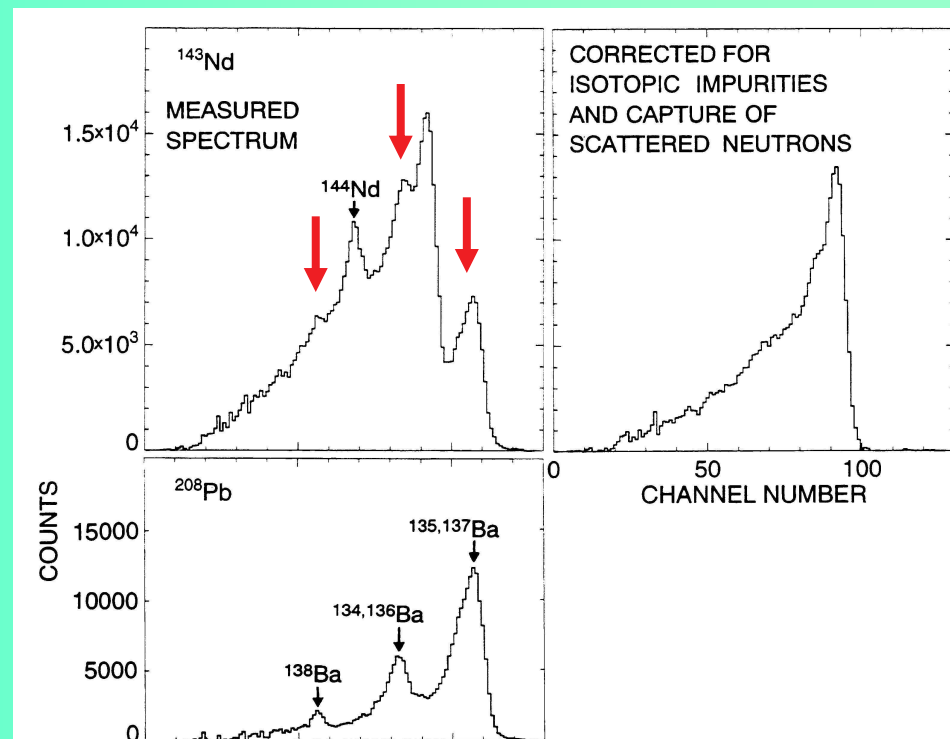
- **What will be the effect of the emitted neutrons?**
- **How can we correct for it?**

- A particular **challenge** is the application of the TAS technique to the measurement of low  $(n,\gamma)$  cross sections, due to the **high scattering/capture ratio**

The sample scattered **neutrons** can be captured in the detector and the subsequently emitted **gamma-rays** become a **contamination source**

→ solution: “subtract” from data

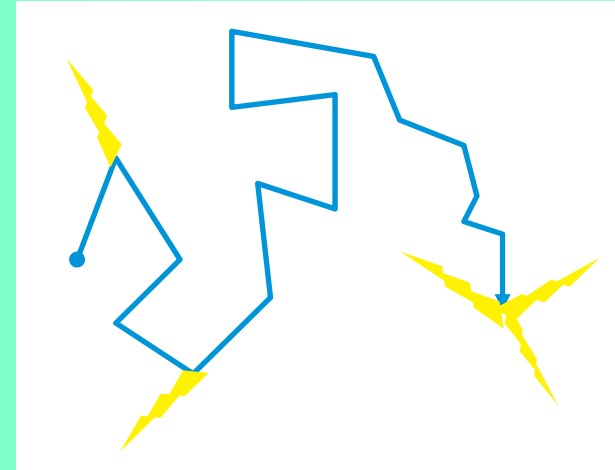
- **Can we obtain a good estimation of the correction?**



TAS @ n\_TOF

**Neutrons** interact through:

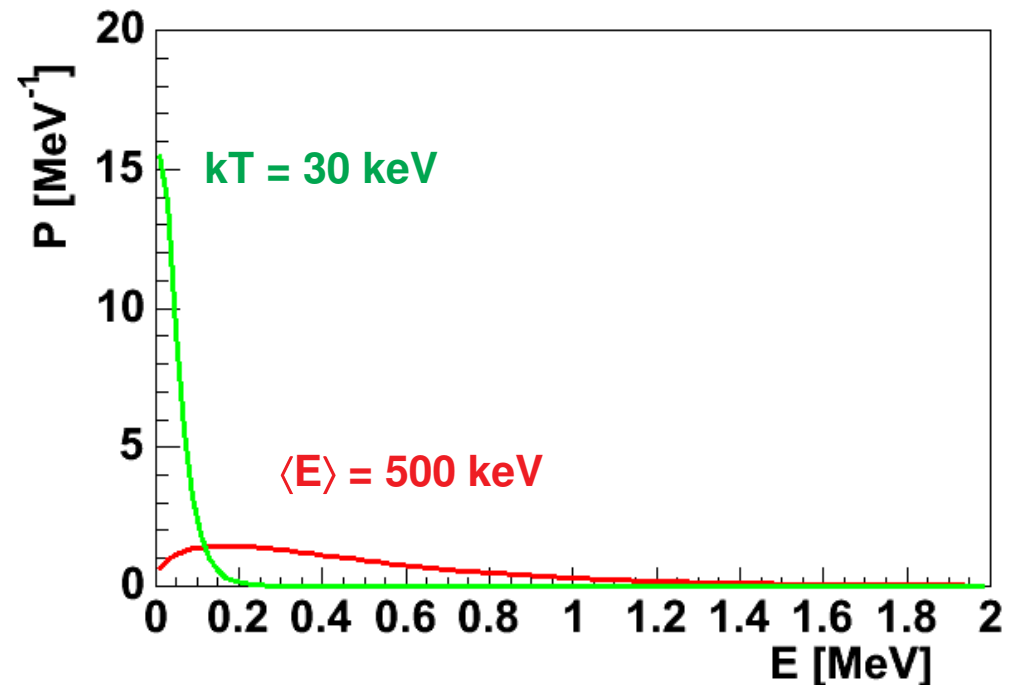
- elastic scattering
- inelastic scattering →  $\gamma$ -rays
- capture →  $\gamma$ -rays
- other: (n,p), (n, $\alpha$ )... →  $\gamma$ -rays

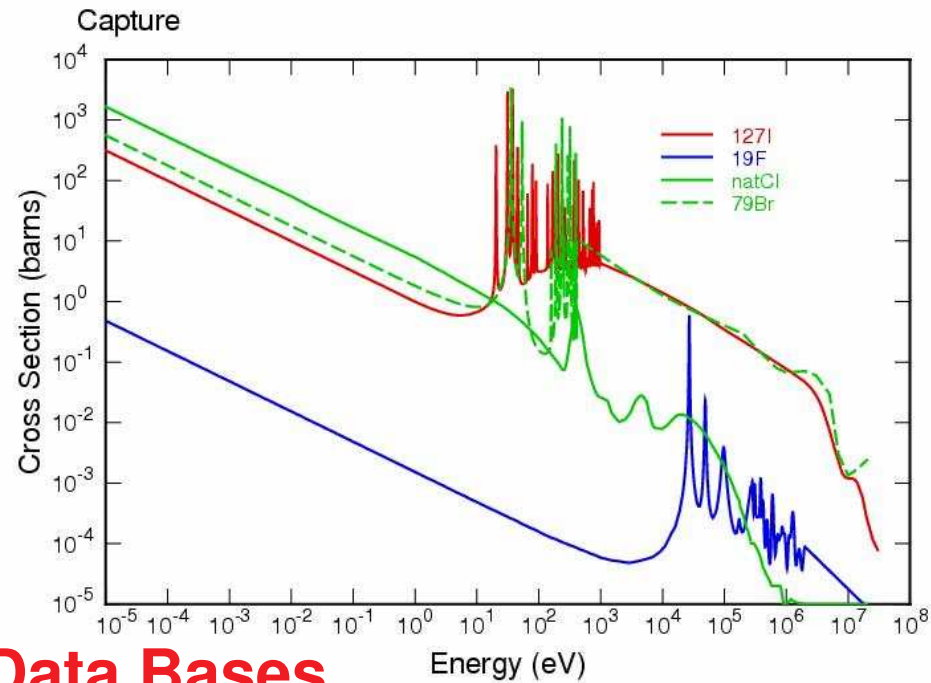
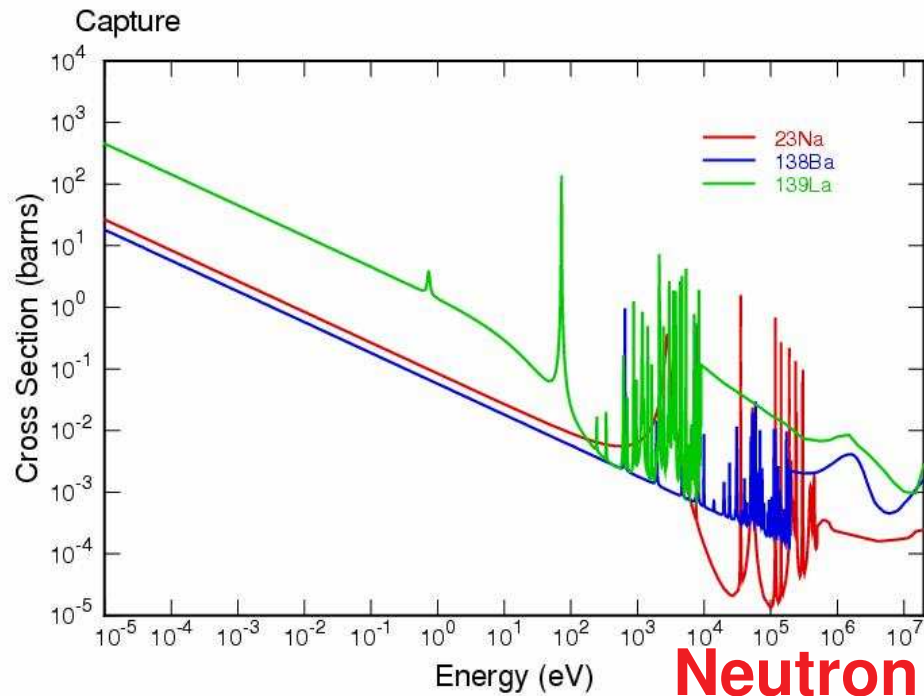


## Neutron energy distribution

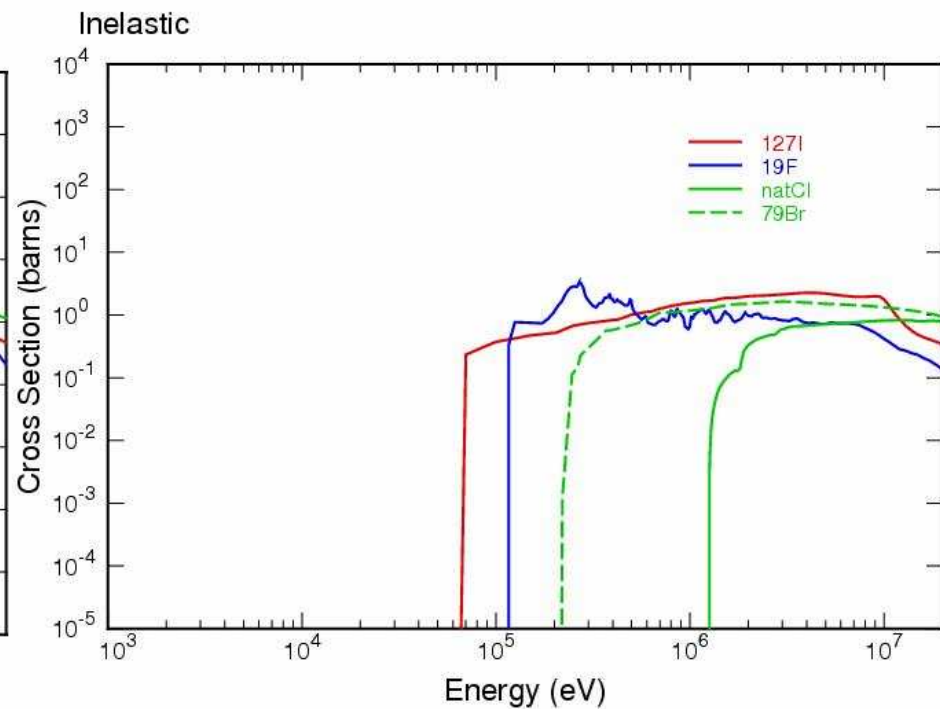
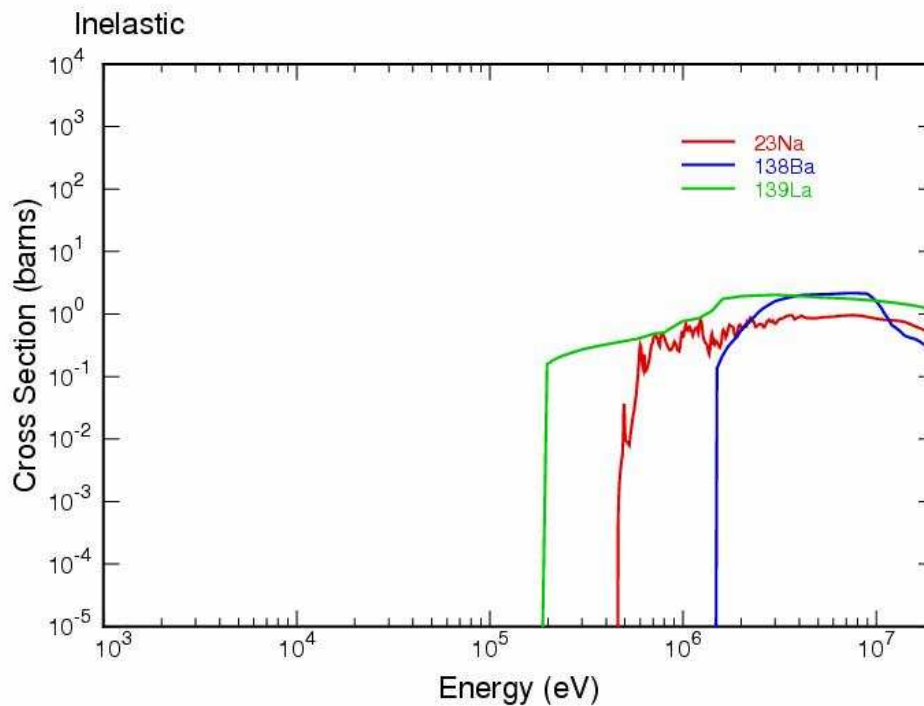
Maxwell distribution of neutrons at a typical stellar temperature  
 $kT = 30 \text{ keV}$

Average energy distribution of delayed neutrons from  $^{235}\text{U}$  thermal fission products:  
~ Maxwell distribution with  $\langle E \rangle = 500 \text{ keV}$





## Neutron Data Bases

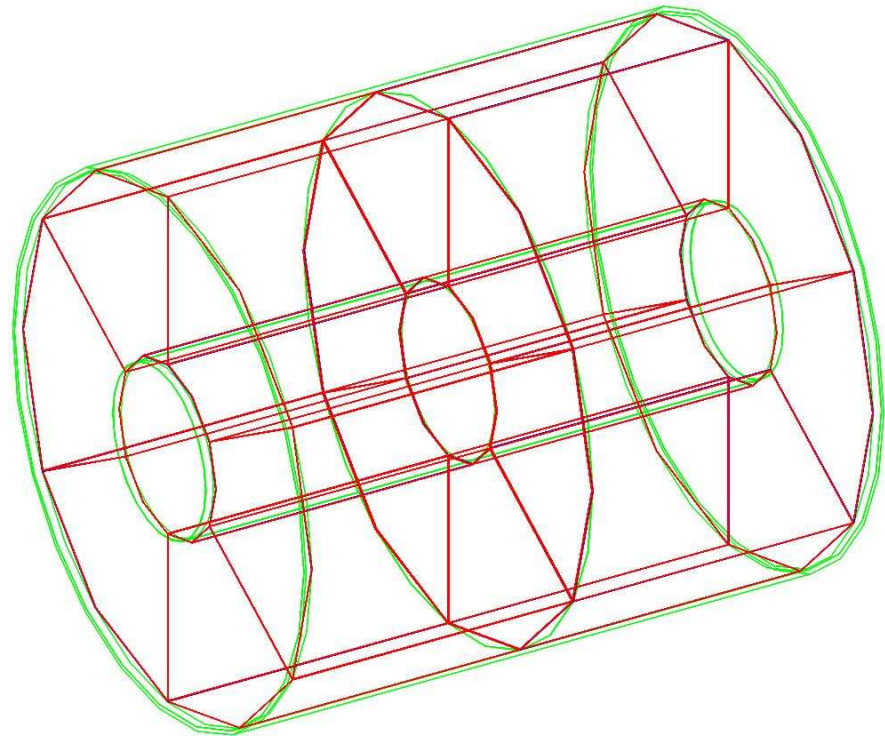


## Investigated scintillation materials

$\sigma$ in mb		CAPTURE				INELASTIC			
MATERIAL	ISOTOPES	$\sigma$ THERMAL	$\sigma$ MXW kT=30keV	$\sigma$ FP <E>=500keV	$S_n$ (MeV)	$\sigma$ THERMAL	$\sigma$ MXW kT=30keV	$\sigma$ FP <E>=500keV	$E_{1stEx}$ (MeV)
Nal(Tl)	<sup>127</sup> I <sup>23</sup> Na	6730	767	205	6.8 6.9	0	183	1053	0.06 0.44
BaF <sub>2</sub>	<sup>19</sup> F <sup>134-138</sup> Ba	1170	57	18	6.6 4.7-9.1	0	328	2350	0.11 0.2-1.44
LaBr <sub>3</sub> (Ce)	<sup>79,81</sup> Br <sup>139</sup> La <sup>140,142</sup> Ce	29700	1335	397	7.9,7.6 5.2 5.4,5.2	0	4	772	0.2 0.17 1.6,0.64
LaCl <sub>3</sub> (Ce)	<sup>35,37</sup> Cl <sup>139</sup> La <sup>140,142</sup> Ce	108300	63	17	8.6,6.1 5.2 5.4,5.2	0	3	378	1.2 0.17 1.6,0.64

## Work started:

- Simulations with Geant4 with recommended hadronic package: **LHEP\_PRECO\_HP**.
- Testing of package is in progress
- Some “problems” detected
- No cross-section data for La, Ce, Br and Cl isotopes



## Simulation geometry:

NaI(Tl):  $R_{in}=10\text{cm}$ ,  $R_{ext}=30\text{cm}$ ,  $L=70\text{cm}$

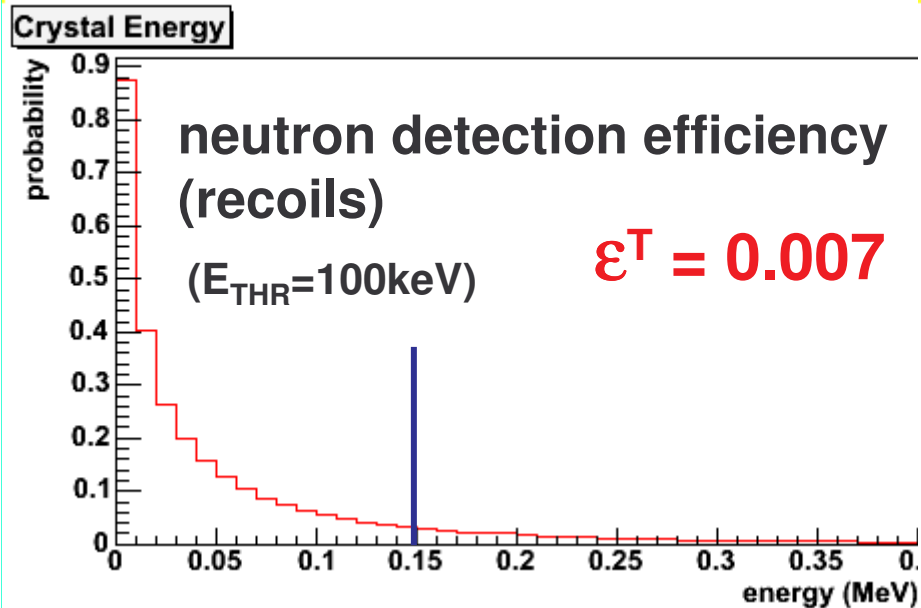
BaF<sub>2</sub> :  $R_{in}=10\text{cm}$ ,  $R_{ext}=25\text{cm}$ ,  $L=60\text{cm}$

Material	$\epsilon^P$ 1MeV	$\epsilon^T$ 1MeV	$\epsilon^P$ 5MeV	$\epsilon^T$ 5MeV
NaI(Tl)	0.85	0.94	0.69	0.89
BaF <sub>2</sub>	0.84	0.92	0.68	0.87

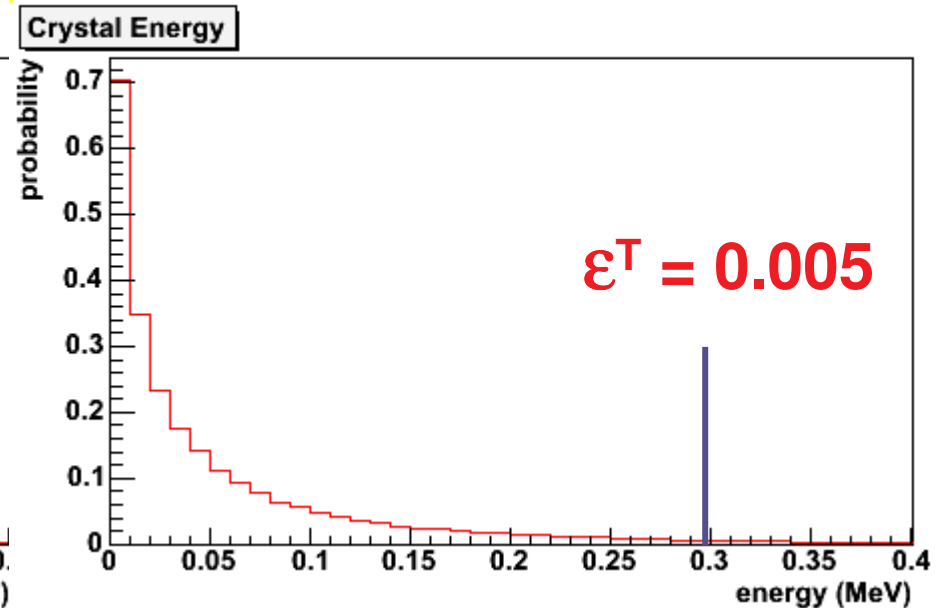


# Energy deposited in TAS by recoils

**NaI, recoils, F(E) distribution**



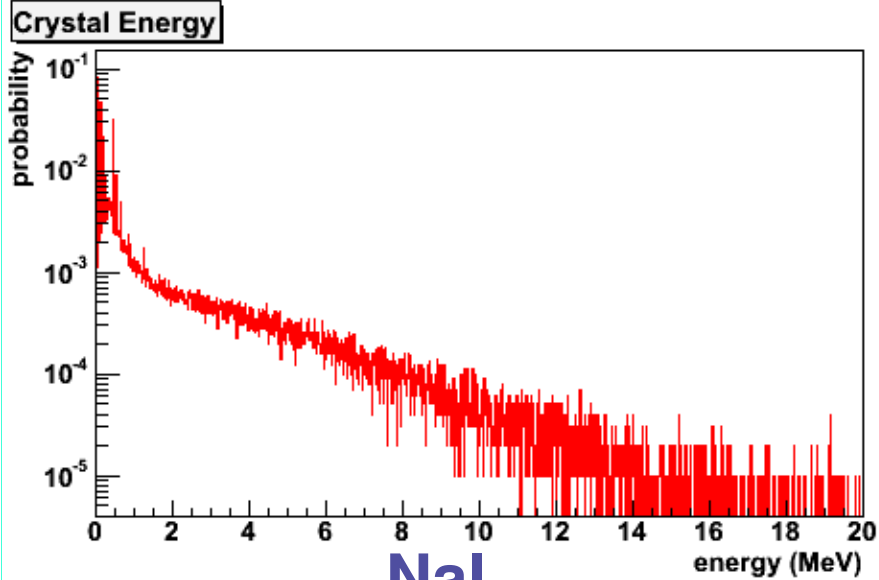
**BaF<sub>2</sub>, recoils, F(E) distribution**



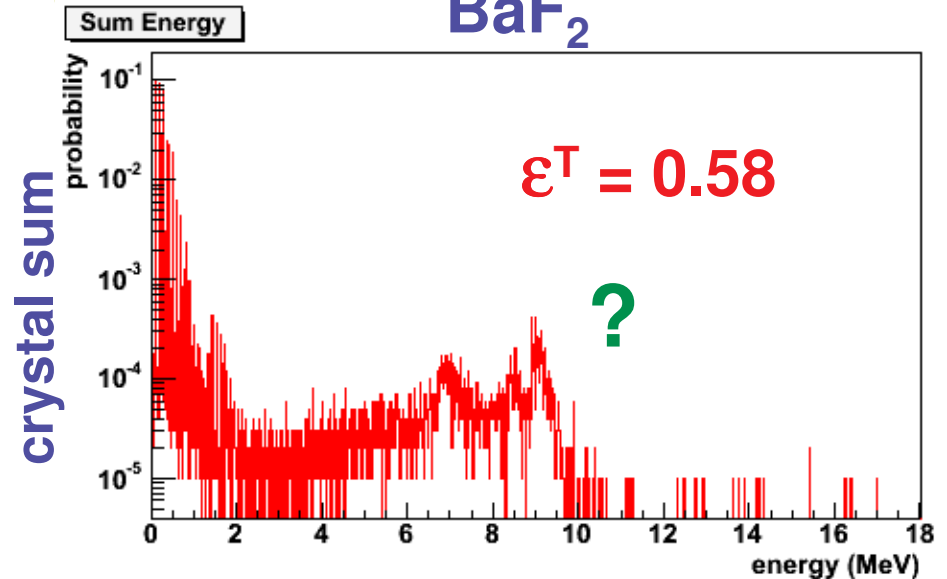
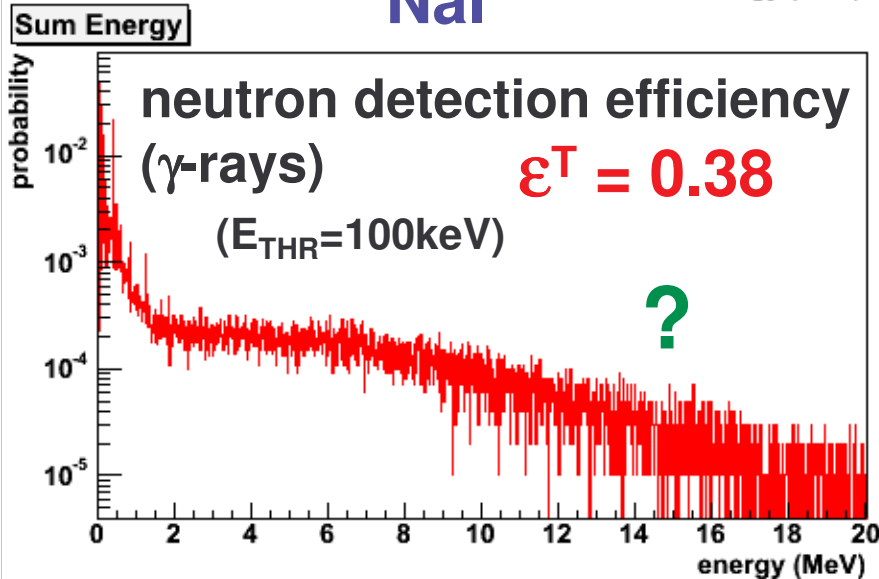
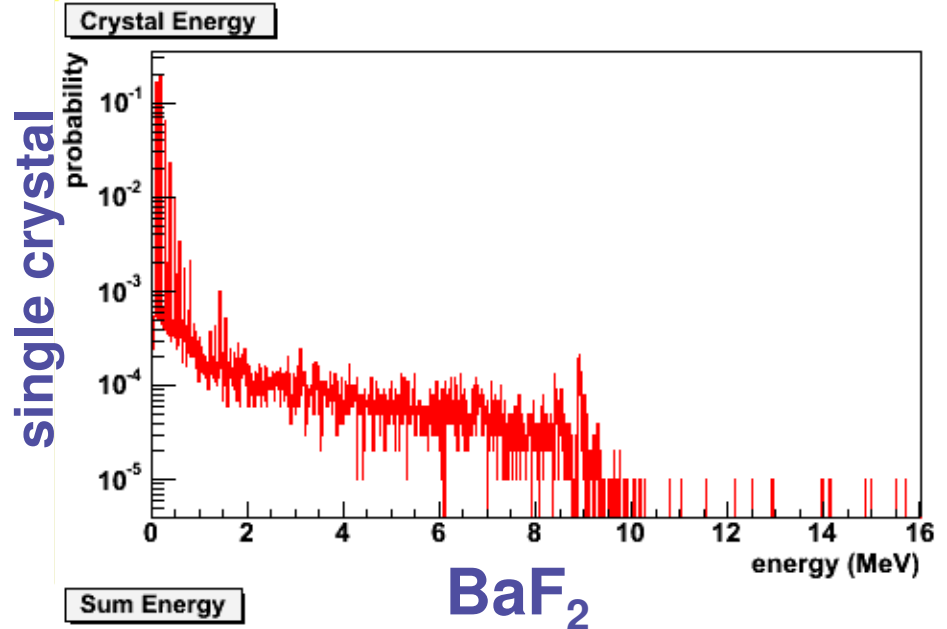


# Energy deposited in TAS by FP delayed neutron energy distribution

Nal, e-, e+ & gammas, F(E) distribution



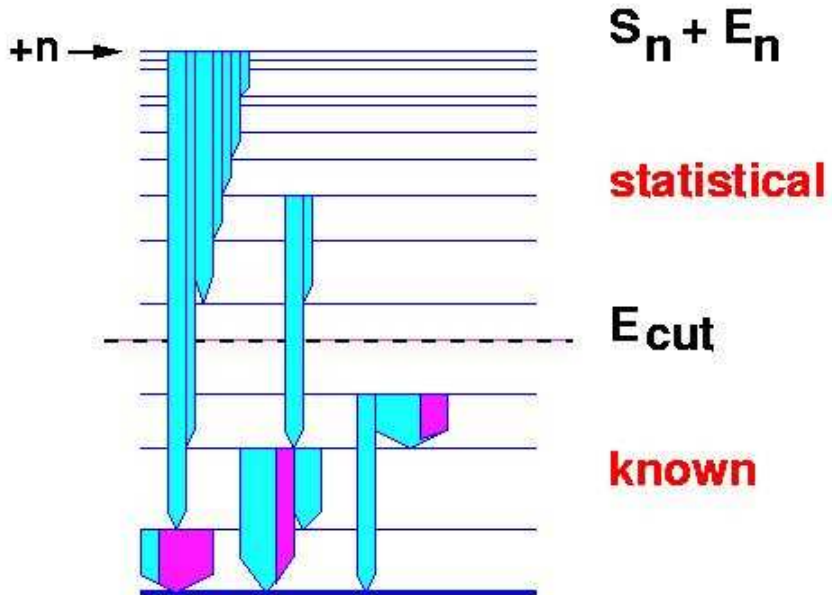
BaF<sub>2</sub>, e-, e+ & gammas, F(E) distribution



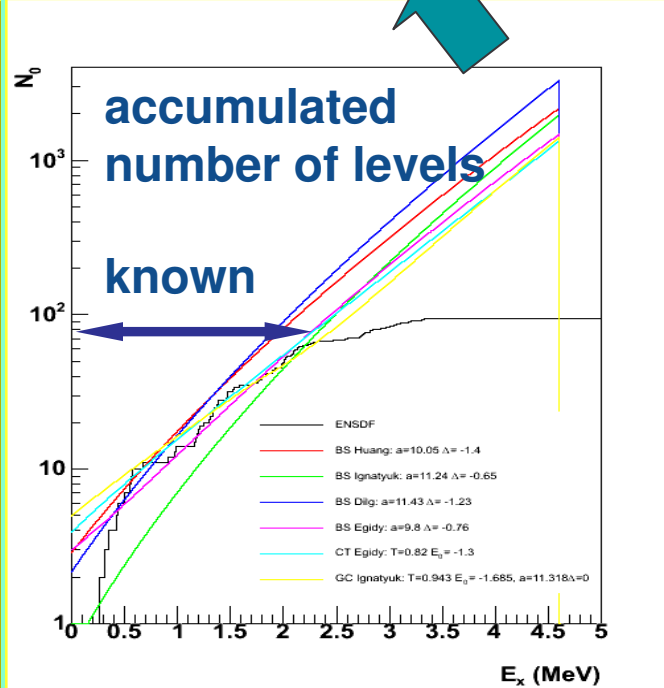
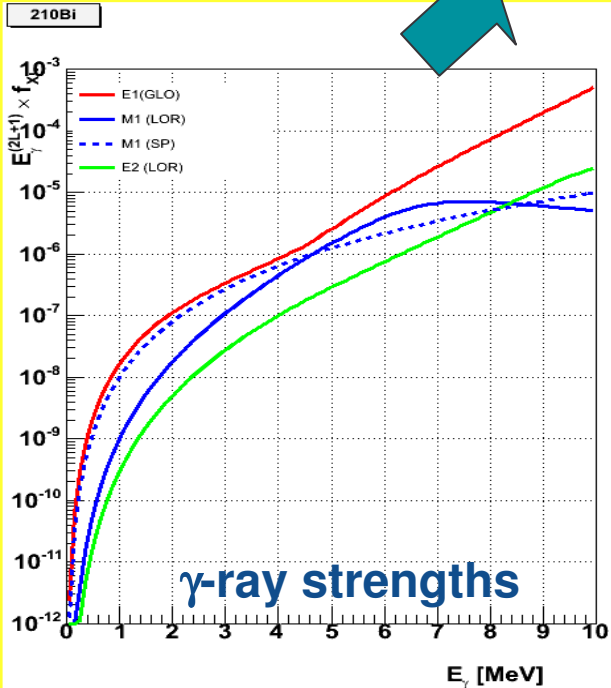
Neutron MC simulation codes have a simplified photon generation

→ Replace by cascades generated with the nuclear statistical model and/or from detailed level schemes

### Statistical cascades

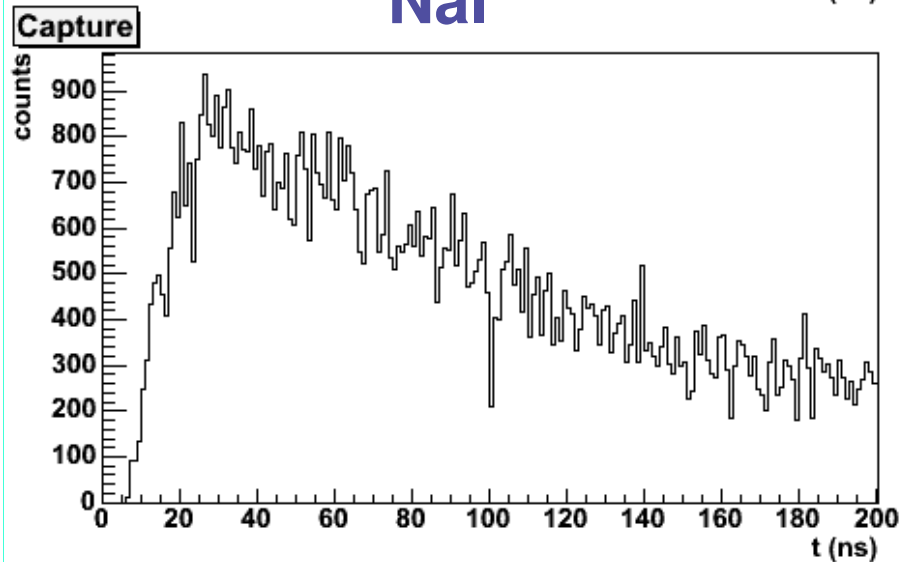
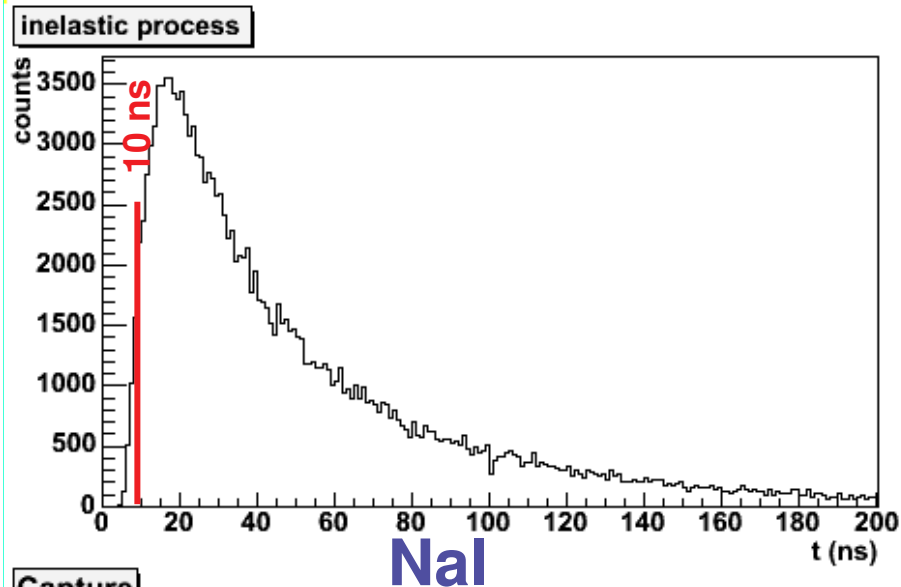


For capture, we have done this work already for GEANT3/GCALOR and D.Cano@CIEMAT for Geant4

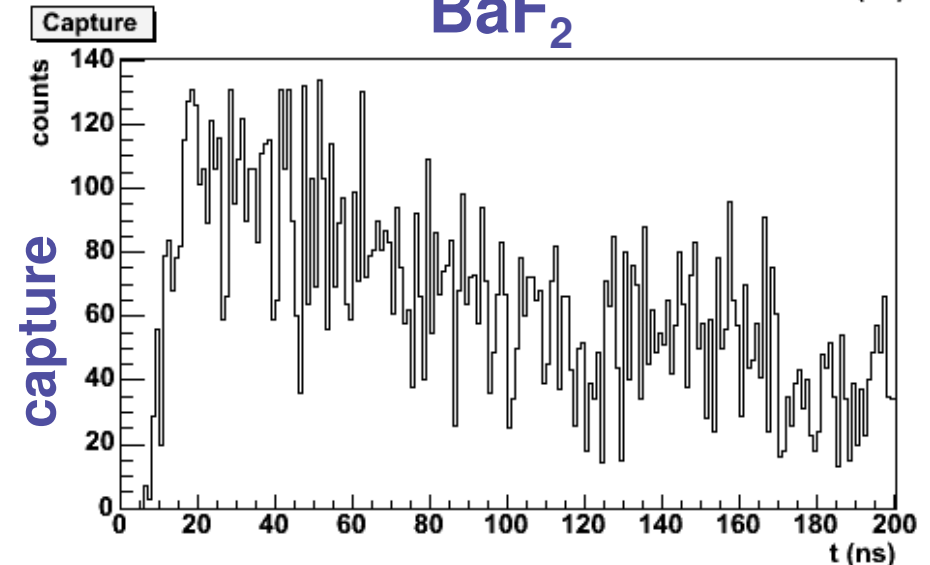
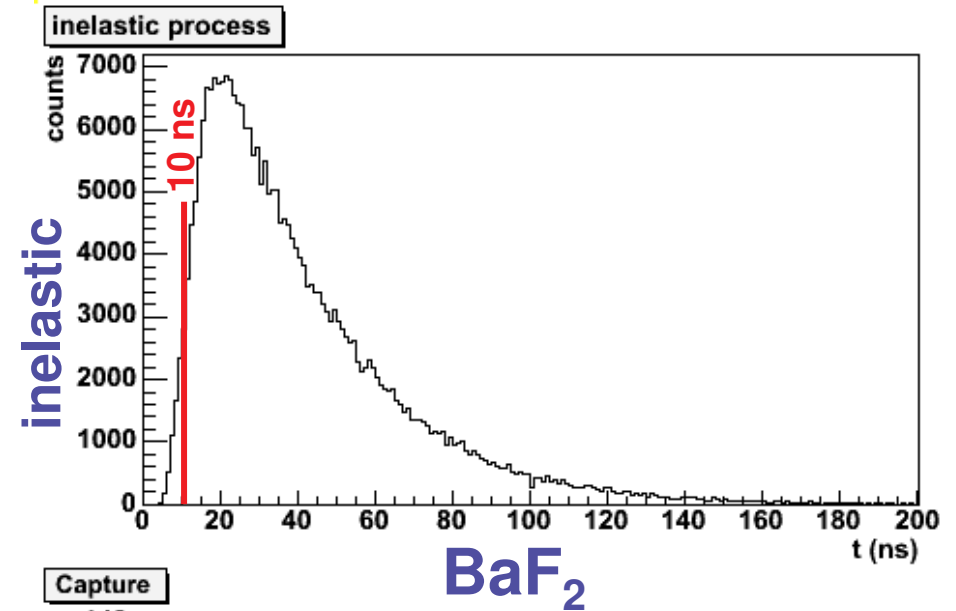


# Interaction time in TAS by FP delayed neutron energy distribution

**Nal ToF, F(E) distribution**



**BaF<sub>2</sub> ToF, F(E) distribution**



## Summary-Conclusions:

- We have presented examples of measurements where an **adequate estimation of low energy neutron interactions** is essential to improve their accuracy
- Although test measurements will be useful, **only MC simulations** can provide the necessary generality
- **Many other fields** (therapy, dosimetry, radiation protection,...) will profit from accurate MC simulations of low energy neutrons
- Although several codes exist, still it is **missing** a general purpose **code that simulates response** to neutrons
- From the very nature of the low energy **neutron data bases** it is essential to allow for **flexibility** in their use
- Although it is not our aim, **we can dedicate some effort**, mainly in testing-debugging if we find some help